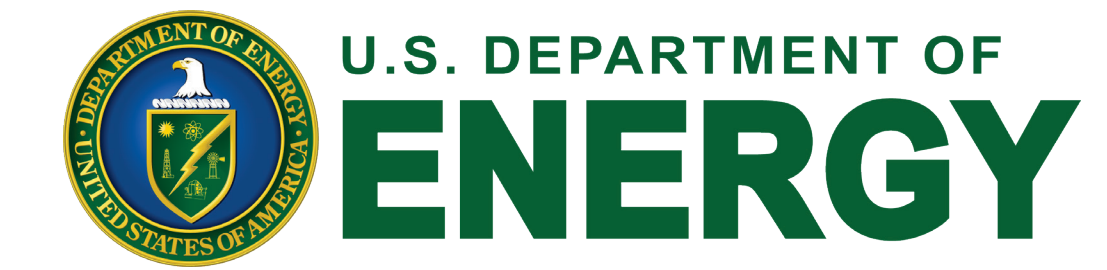


# Verification and Validation of COMSOL Multiphysics to Support HFIR Applications

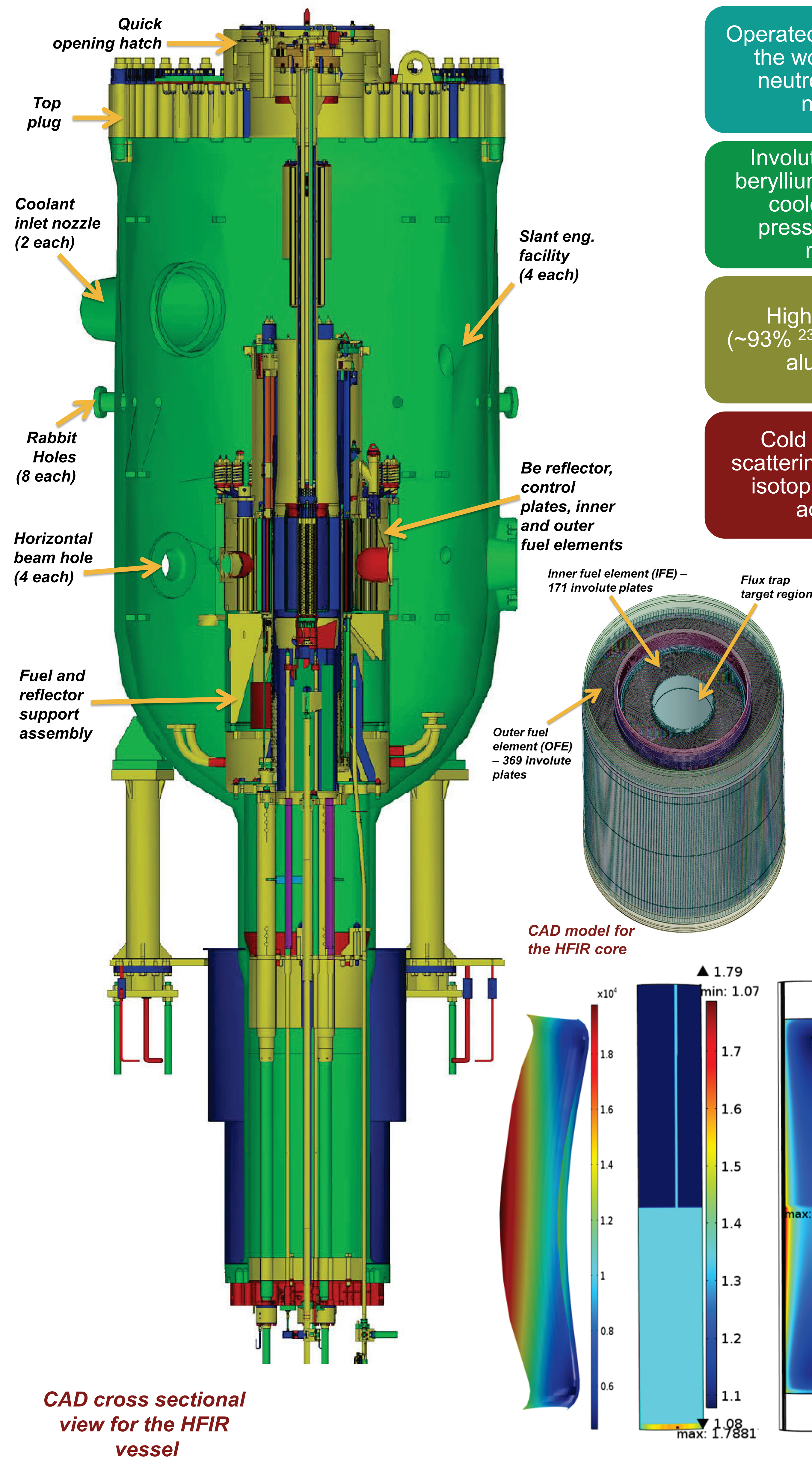


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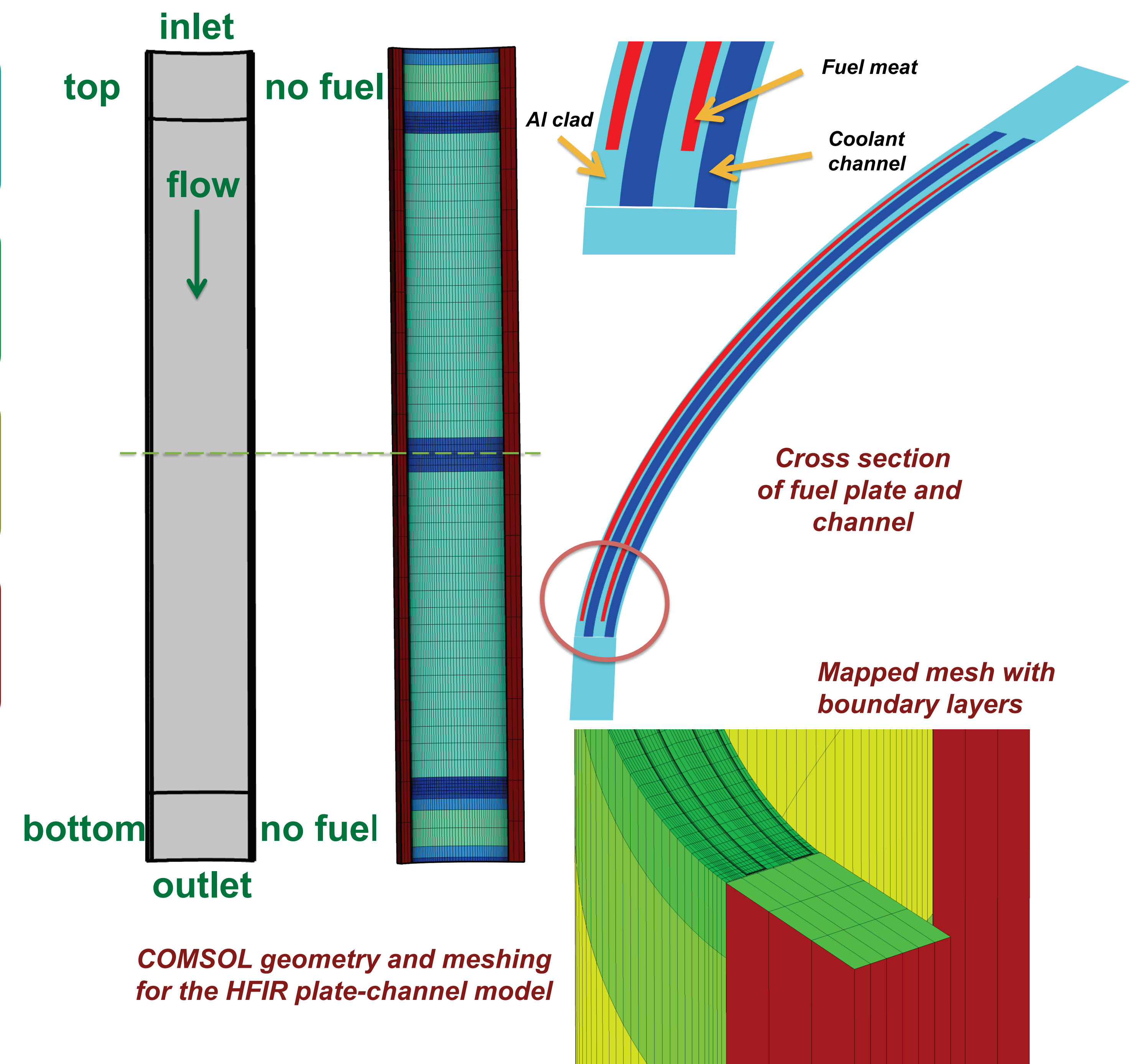


## High Flux Isotope Reactor (HFIR)

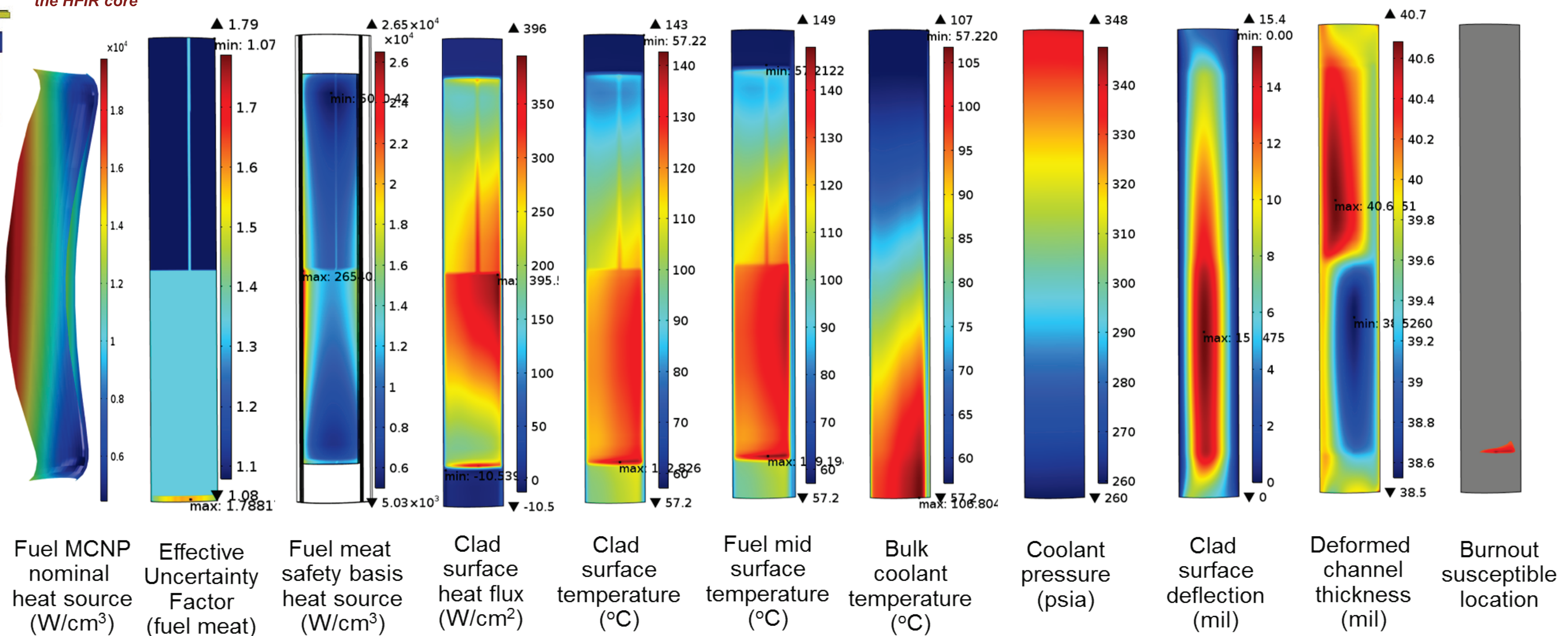
## COMSOL Thermal-Hydraulics Models for the HFIR Core



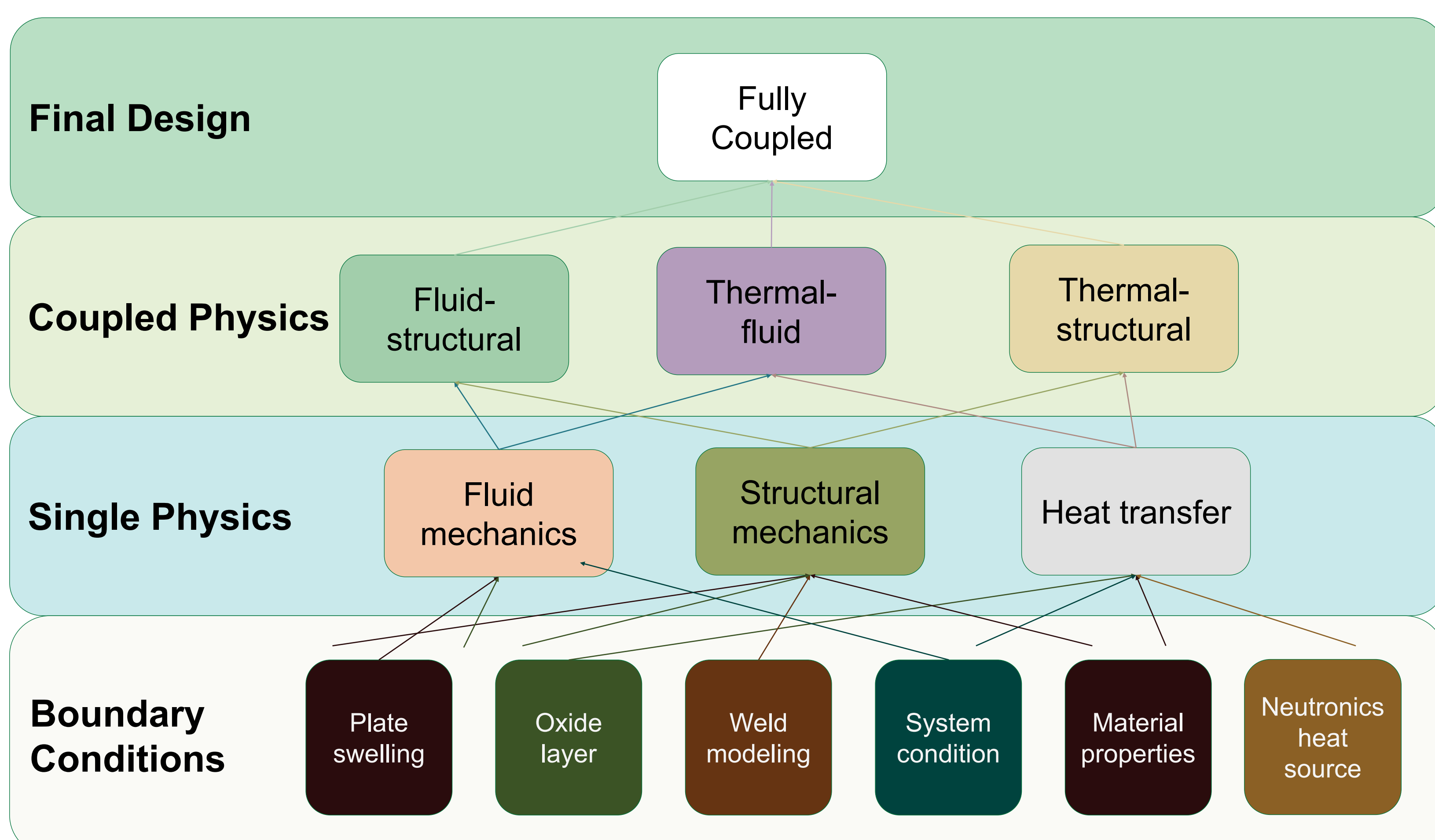
- Operated since 1966 with one of the world's highest thermal neutron fluxes  $\sim 2.5 \times 10^{15}$  neutrons/(cm<sup>2</sup>-s)
- Involute-shaped fuel plates, beryllium reflected, light water-cooled and -moderated, pressurized, flux-trap type research reactor
- Highly enriched uranium ( $\sim 93\%$  <sup>235</sup>U/U) fuel embedded in aluminum-6061 clad
- Cold and thermal neutron scattering, materials irradiation, isotope production, neutron activation analysis



## COMSOL Results for the HFIR HEU Inner Fuel Element under Hypothetical Safety Basis Conditions



## Hierarchy of Verification and Validation (V&V)



## Select V&V Problems for HFIR

Physics	Validation comparison
HFIR system	HFIR plant data
Fuel swelling	Advanced Test Reactor (ATR) data
Oxidation	American Nuclear Society (SNS) data
Structural attachment	HFIR plate pull test data
Heat transfer	Analytic solution of heat transfer
Fluid mechanics	National Advisory Committee for Aeronautics (NACA) experiments
Structural mechanics	Analytic solution of plate deflection
Fluid-thermal	STAR-CCM+ comparison, Gambill & Bundy
Fluid-structural	Smislaert experiments
Thermal-structural	Cheverton Kelly